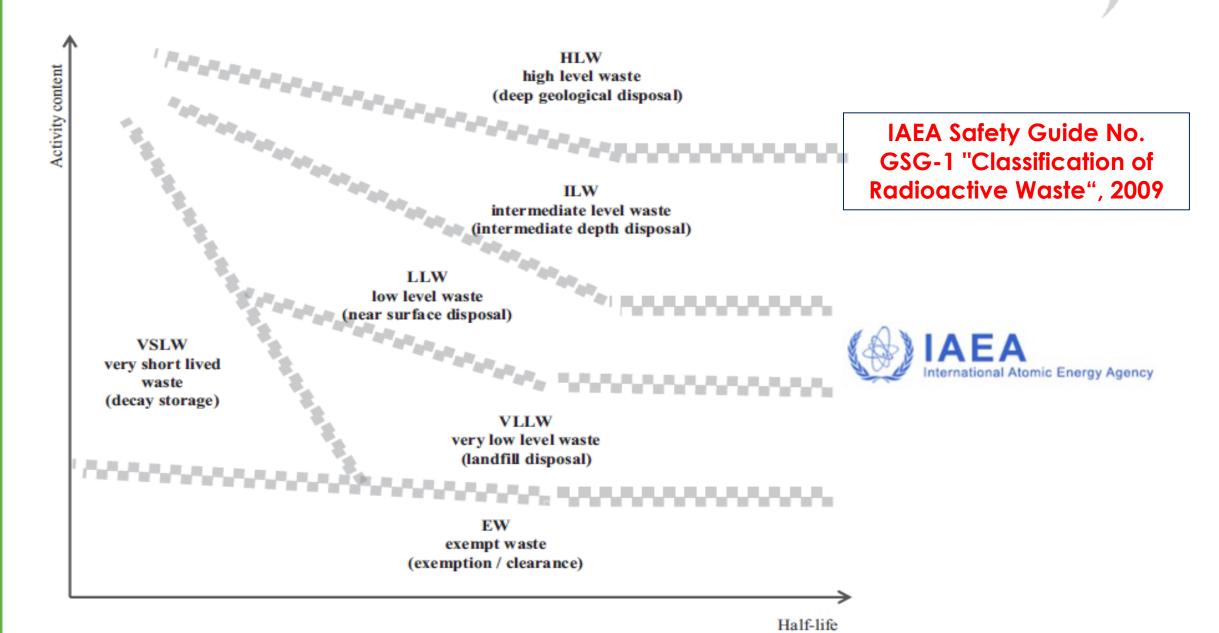
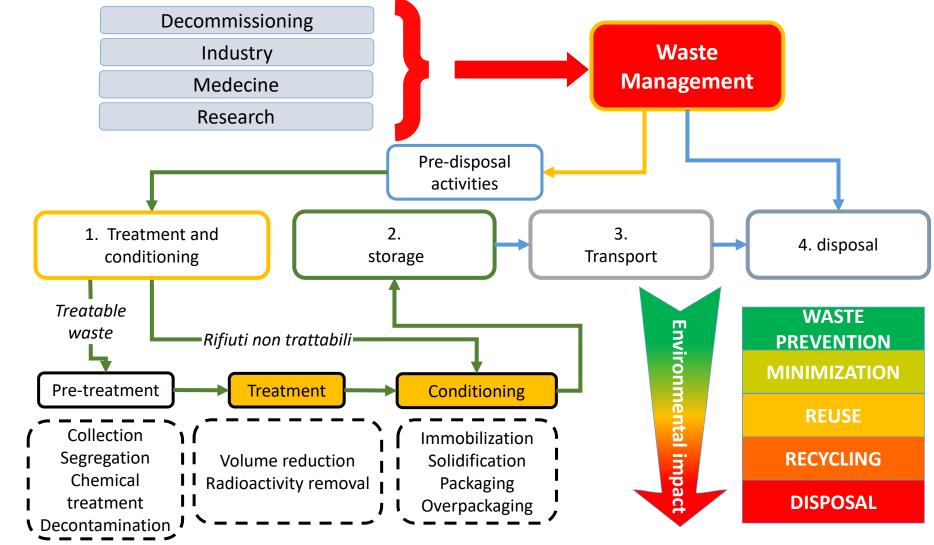


Annafrancesca MARIANI

Radioactive waste IAEA classification



Management of radioacive waste and materials



Security Class: Public Use

Management of materials and waste

SOGIN

Material classification and Flow diagrams



Materials management is a key issue of the decommissioning process. Based on their origin and radioactivity level, the materials resulting from decommissioning activities may be classified in:

CONVENTIONAL

Materials from conventional areas of the plant. Sample radiometric checks before leaving the site.

REMOVABLE/RELEASABLE

Materials from areas with radiological impact. Radiometric checks in order to verify that the radioactivity levels comply with the "clearance level".

Components or materials may be re-used, recycled or disposed of as conventional materials.

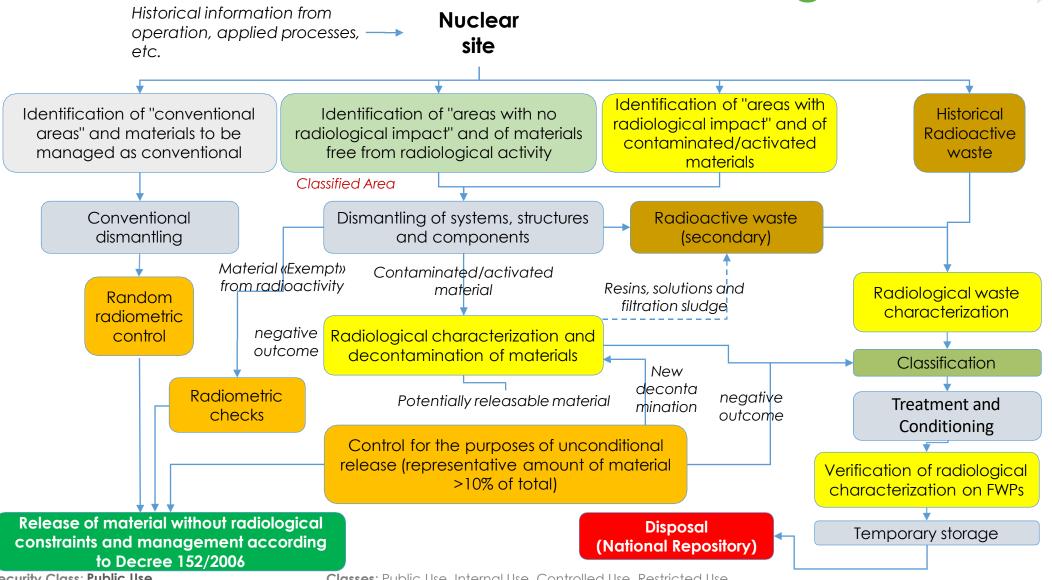
EXEMPT

Materials from exempt areas of the plant. Radiometric checks in order to verify the absence of radioactive contamination. The materials are removed from the site free of radiological risks.

NON-RELEASABLE MATERIALS

Materials from areas with radiological impact. Materials to be managed as radioactive waste.

Material classification and Flow diagrams



Security Class: Public Use

Decommissioning Waste inventory



AUTHORISED LANDFILL

anagement of materials and waste



Specific Principles on RWM



The specific principles in Radioactive waste management can be grouped into the four following fundamental specific principles:

- **1. Avoid generation of radioactive waste** in other word keep the generation of radioactive waste to the minimum possible or practicable;
- 2. Minimize the spread of radioactivity by containing it to the greatest extent possible. That means for example using a compartment strategy in line with the different contamination levels and prefer additional compartment for the most contaminated areas;
- **3. Minimize the amount of radioactive waste** by applying adequate treatment technology, for example sorting or supercompaction;
- **4. Optimize possibilities for recycle and reuse** of valuable components from existing and potential waste streams;

These specific principles are derived from IAEA TRS 401.

Waste reduction – SUPER-COMPACTION

A hydraulic press is used which works between 1000 and 2000 tons (order of hundreds of MPa) by vertically compressing the 220 liter metal drums containing radioactive waste.

The volume reduction factors achieved are typically in the range of 2-10 depending on the type of material

The compacted pellets are then placed in overpacks for subsequent conditioning













Security Class: Public Use

Thermal treatment – FUSION

This is a technology widely used in the nuclear sector and allows for the recycling of more than 95% of the initial material

Radioactivity is concentrated in the melt slag (<5% of initial volume)



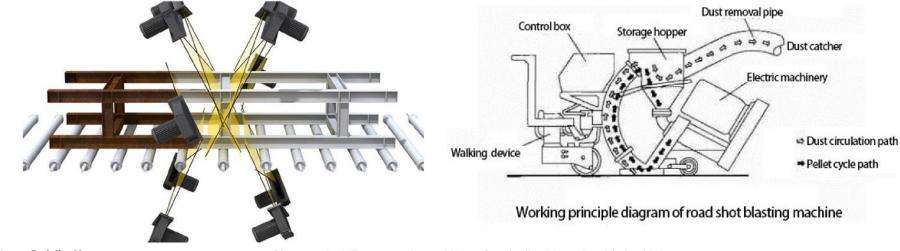


DECONTAMINATION AFTER DISMANTLING

Use of abrasive materials (metal balls or sand) to remove the surface contamination in order to reduce the final amount of waste.

The removed surface material and the abrasive are collected and managed as radioactive waste, the recirculation of the abrasives allows secondary waste to be minimized.

Applicable to most materials except those that may be crushed from the abrasive (e.g. glass or plexiglass) It is most effective on flat surfaces and because the abrasive is sprayed, it is also applicable to areas difficult to reach



Security Class: Public Use

"ENCAPSULATION"

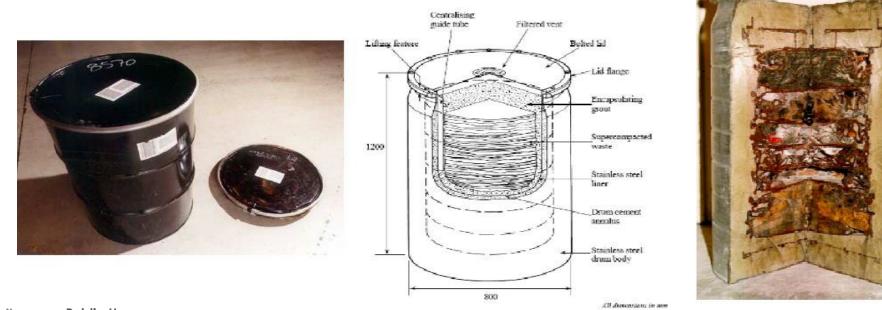
SOGIN

Encapsulation is the process of immobilizing solid waste by applying a fluid matrix (often cement-based) that completely penetrates the waste to form a solid monolith



"EMBEDDING"

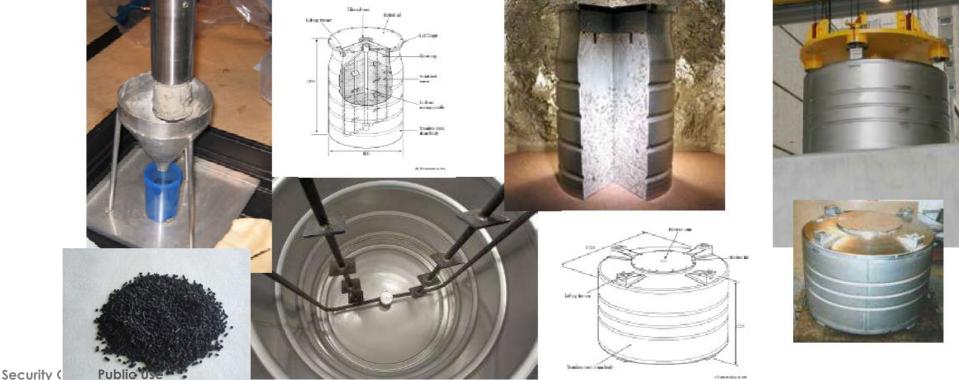
Embedding is a typical solid waste immobilization process, where the waste is incorporated into clay, mortar, sand, cement, etc. Generally, the elements (or additives) are inserted in the form of a fluid matrix together with the waste in a container, so that the voids can be filled and form a solid monolith. This process is generally used to immobilize waste in cylindrical containers and/or prismatic containers ("large containers").



שומשפש. דטטוול טשב, ודובודותו טשב, לטרוונטוובת טשב, ולבשווולובת טשב

SOLIDIFICATION

"Solidification" is the immobilization process of non-solid phase waste (generally liquid or semi-liquid waste) which is generally applied by mixing the waste with cement, additives and water. A disposable stirrer is used to homogenize the mixture. The cement is then dried as for the "encapsulation process" to form a solid monolith.





VETRIFICATION

High activity liquid radioactive waste (fission products obtained from the reprocessing phases of the irradiated fuel) is mixed with molten glass and poured into stainless steel containers (canisters) in order to obtain a solid matrix (vitrified waste).

Glass guarantees chemical/physical stability for long periods and is characterized by high resistance to leaching, radioactivity and heat; it therefore constitutes an excellent matrix for the containment of radioactive waste.

fission-product solution resistance furnace glass frit particle separator rotating kiln canister





(1)

Italian waste management strategy

- Treating and conditioning all liquid and solid waste for delivery to the National Repository to be realised in Italy
- **Transport abroad** of the Spent Nuclear Fuel for reprocessing (except particular cases)
- **Receiving back** to Italy the residues foreseen by the reprocessing contracts
- Interim storage of waste at the production sites
- **Disposal** of Very Low and Low Level Waste in the National Repository
- Long-term storage of Medium and High Level Waste in the National Repository pending the availability of a geologic repository
- Implementing R&D aimed at the safe management of Spent Nuclear Fuel and Radioactive Waste
- Transparency and actual participation by the public in the decision process relevant to SNF and RW management

Storage of operational waste

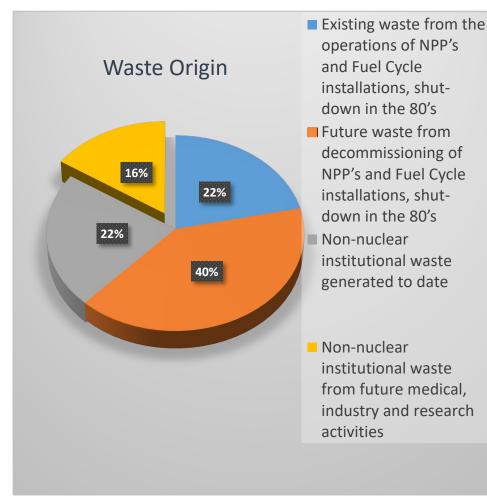
Currently the whole quantity of waste generated during the decommissining operation of the Sogin nuclear installations is safely stored at the same sites and partially at the Nucleco premises

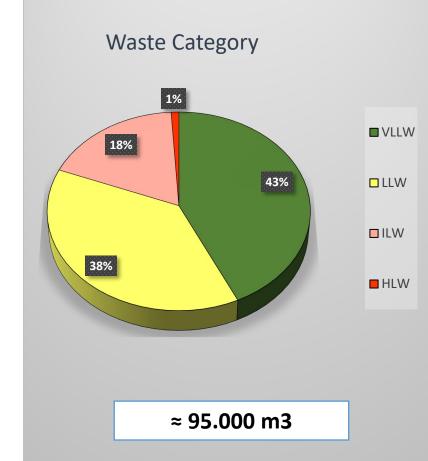




Radwaste inventory in Italy (Forecast of the overall volumes to be disposed of*)

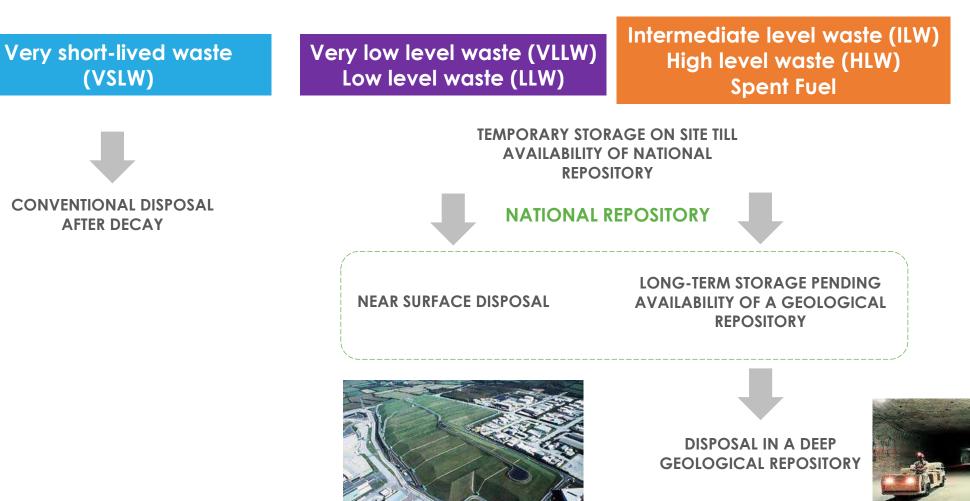






Security Class: Public Use

DISPOSAL STRATEGY IN ITALY



Security Class: Public Use

CURRENT WASTE STORES IN ITALY

Waste from medicine, research, industry

- VSLW managed by institutional producers, like hospitals, until natural decay allows their disposal as conventional (special) waste
- remaining LLW and ILW are collected by the operators of the so-called Integrated Service and stored in temporary storage facilities pending availability of the National Repository

Waste from nuclear power plants, fuel cycle facilities, research reactors and decommissioning

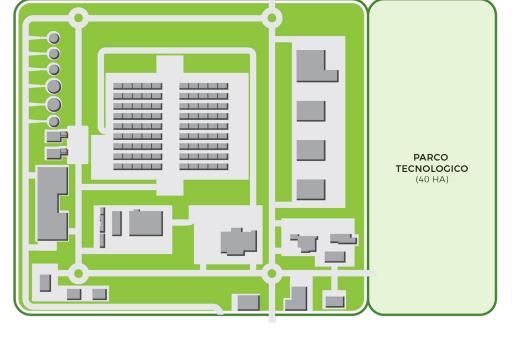
 stored by Sogin in temporary storage facilities at the sites of production pending availability of the National Repository





NATIONAL REPOSITORY AND TECHNOLOGY PARK

- The National Repository is a surface infrastructure designed to place **all Italian radioactive waste**
- The project also envisages creating a Technology Park, a **research centre** in the fields of energy, waste management and sustainability
- Law No. 31 of 2010 entrusted Sogin with the task of siting, designing, building and operating the National Repository and Technology Park
- For the localization of the site in which the infrastructure will be built, Sogin has managed a public consultation, lasted about a year (5th January 2021 - 14th January 2022), which included a National Seminar
- It represented an **efficient tool for civil society stakeholders' engagement** (institutions, associations, committees, companies, professionals and citizens): **over 600** questions, observations and proposals from 322 subjects



DEPOSITO NAZIONALE (110 HA)



The National Repository will collect about 95.000 m³ of radioactive waste over time: 60% of waste results from the nuclear decommissioning, while 40% from scientific research, medical applications and industry.



SITING CRITERIA



Technical Guide n.29 published in June 2014 by ISPRA (Italian Regulator) with siting criteria for a near surface disposal facility

Two categories of criteria are defined:



- **15 Exclusion Criteria** Applied by means of data on a national scale they led to the exclusion of areas whose characteristics do not comply with the main safety principles, resulting in the identification of potentially suitable areas
- 13 Investigation Criteria Applied to the not excluded areas by means of available data on a local scale for further exclusion of unsuitable areas; they will be useful for elaborating a suitability order of potentially eligible areas and for the characterization of the sites of interest.

EXCLUSION CRITERIA

| with presence of active or quiescent volcanoes |
|---|
| with high seismic activity |
| interested by superficial faulting |
| characterized by geomorphological and/or hydraulic risk and/or hazard of any grade as well as river belts |
| with holocene alluvial deposits |
| located above 700 m a.s.l. |
| characterized by an average slope greater than 10% |
| within 5 km from the current coast line or, if more distant, located under 20 m a.s.l. |
| interested by morphogenetic karst processes or with presence of sinkholes |
| with near surface piezometric levels or with piezometric levels which could anyhow interfere with the foundation of the disposal facility |
| naturalistic, protected under the legislation in force |
| at a unsuitable distance from residential zones |
| within a distance of 1 km from highways, all principal suburban roads, and the main and complementary railway lines |
| with known presence of underground resources |
| with industrial activities involving major accident hazards, dams and artificial hydraulic barriers, airports or operating military shooting ranges |

EC1

EC2

EC3

EC4

EC5

EC6

EC7

EC8

EC9

EC10

EC11

EC12

EC13

EC14

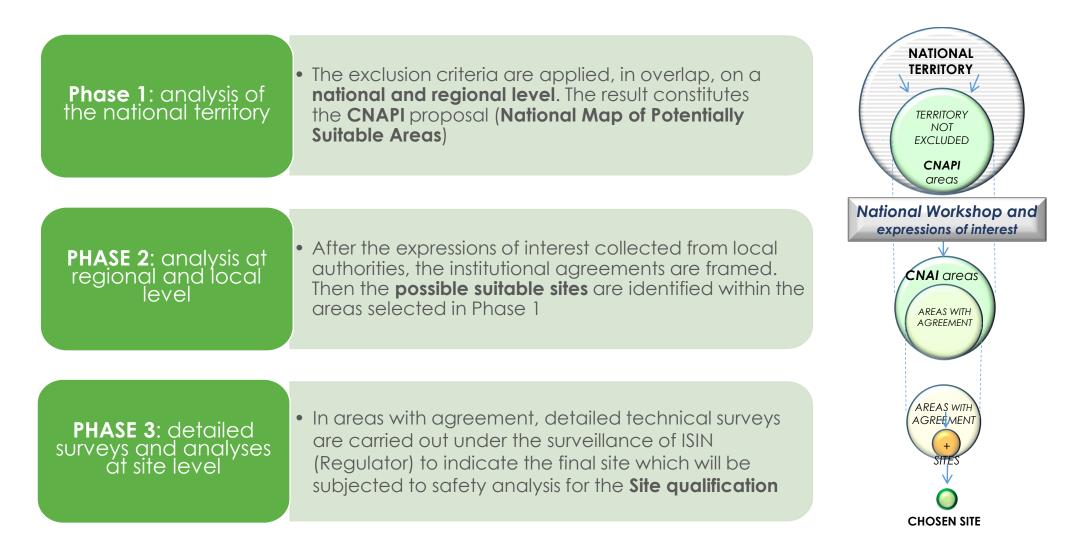
EC15

| IC1 | presence of secondary volcanic activities |
|------|---|
| IC2 | presence of significant vertical movements as a result of subsidence and uplift phenomena (tectonic and/or isostatic) |
| IC3 | geological-morphostructural setting and presence of lithotypes with vertical and lateral variation |
| IC4 | presence of endorheic type river basins |
| IC5 | presence of accelerated erosion phenomena |
| IC6 | weather and climatic conditions |
| IC7 | physical and mechanical parameters of the soil |
| IC8 | hydrogeological parameters |
| IC9 | chemical parameters of soil and groundwater |
| IC10 | Habitats, animal and plant species of conservation importance, as well as geosites |
| IC11 | agricultural production of outstanding quality and places of archaeological and historical interest |
| IC12 | availability of primary transport infrastructures |
| IC13 | presence of relevant or strategic critical infrastructures |

SOGIN

SITING PHASES

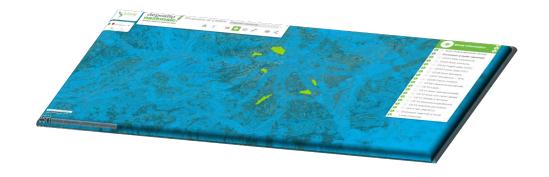




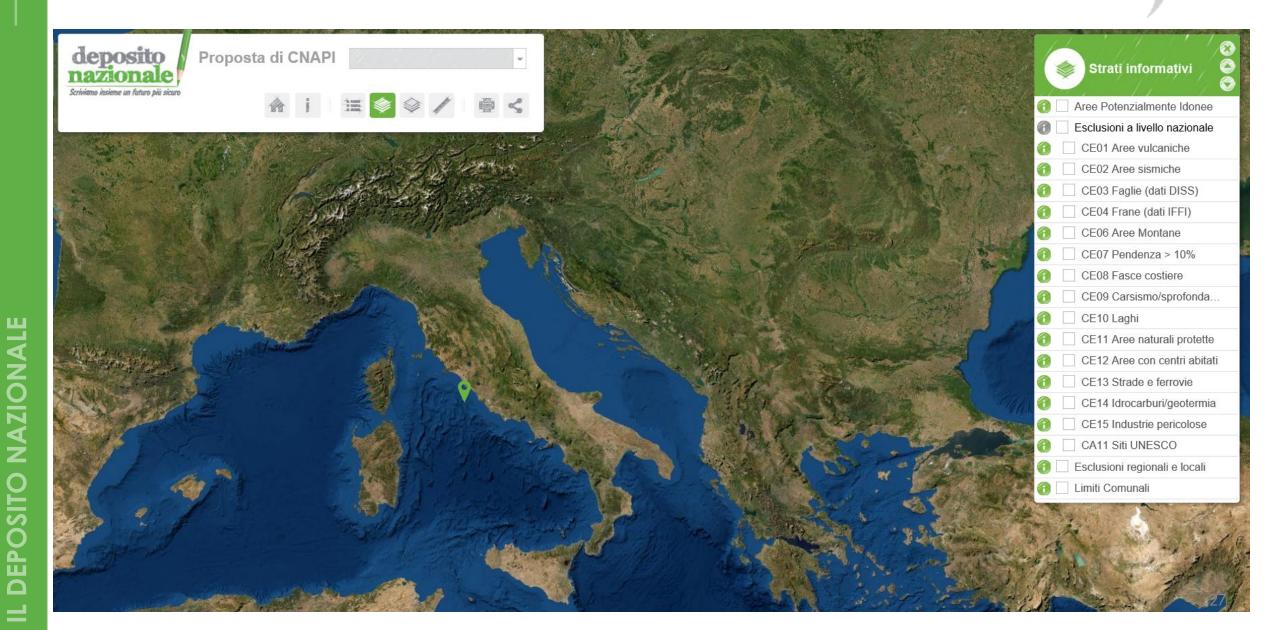
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Sogin applied a procedure in two separate phases, as suggested by IAEA (SSG – 29):

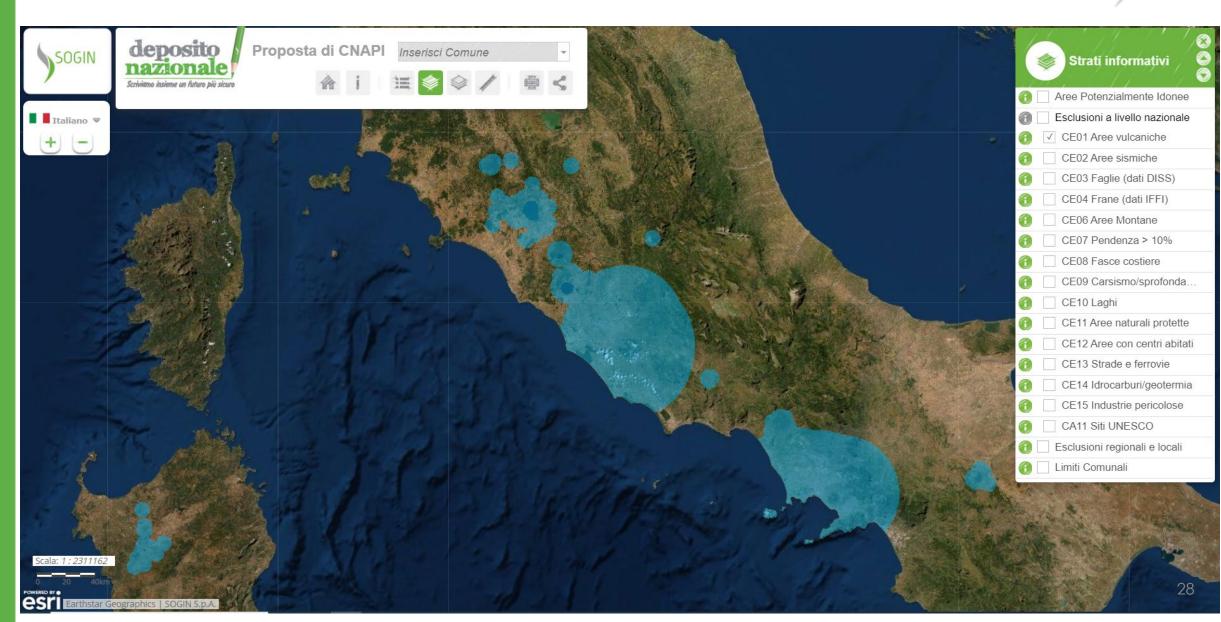
- Regional mapping phase: areas are excluded if the analysis of the available data shows their non compliance with the Exclusion Criteria defined by ISPRA
- Site screening phase: areas are excluded or ranked according to more detailed data compared to the Investigation Criteria defined by ISPRA



The "potentially suitable areas" are areas, even vast, which have characteristics favorable to the identification of sites suitable for the localization of the repository



SOGIN



SOGIN

NATIONAL REPOSITORY



SOGIN



ATIONAL REPOSITORY

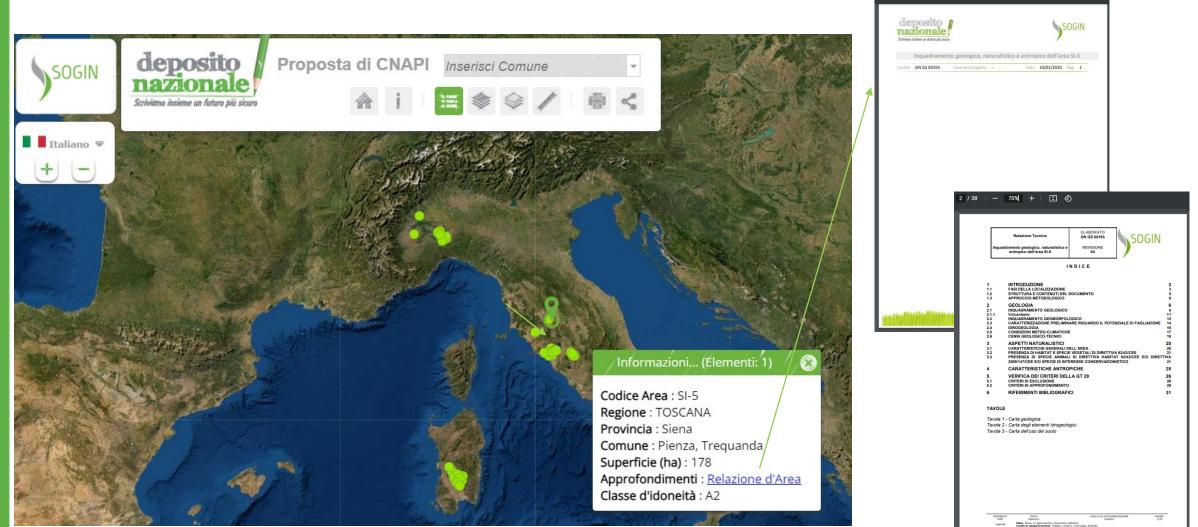
NATIONAL REPOSITORY



National territory excluded 99,93% 29.978.212 ha

deposito nazionale Proposta di CNAPI Inserisci Comune SOGIN 1 1000 Legenda Scriviamo insieme un futuro più sicuro Aree potenzialmente idonee Italiano 👳 +

31

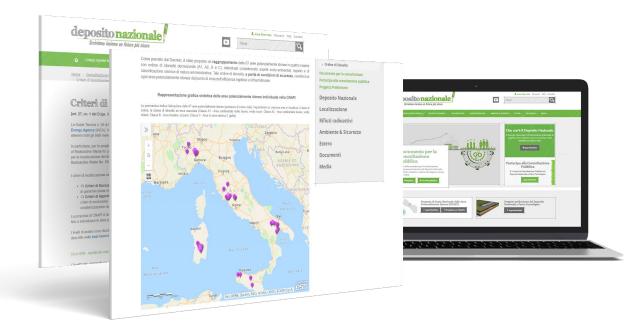


/38 | - 71% + | 🗄 🔊





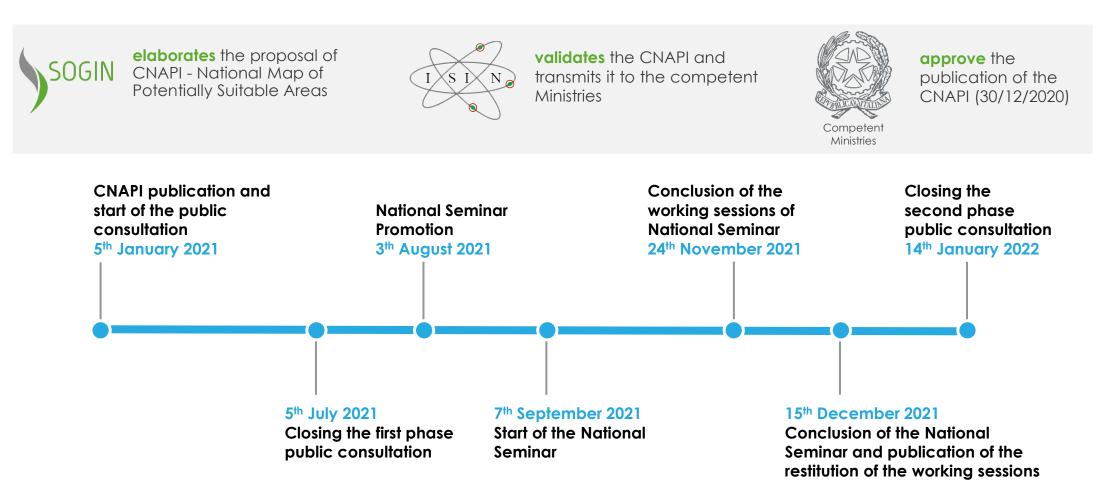
67 Potentially suitable areas, 7 Regions



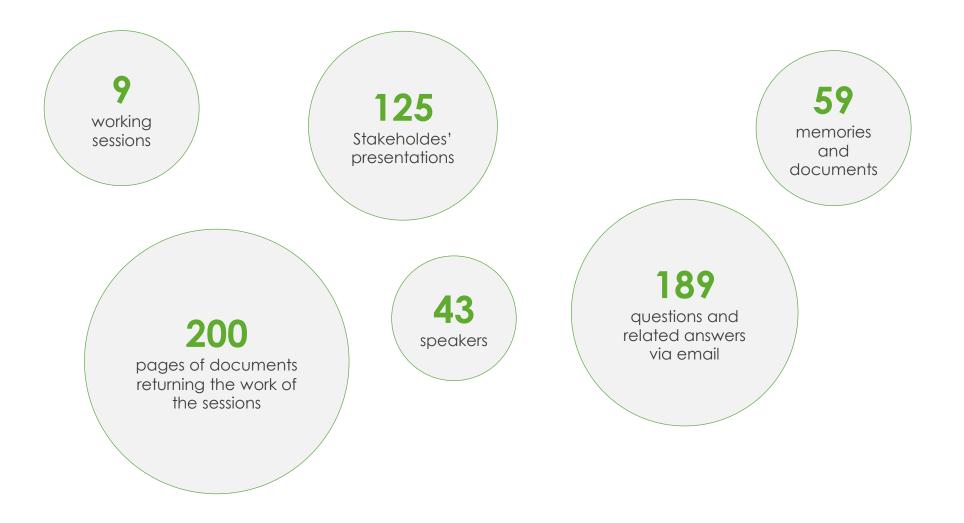
- Piedmont: 8 areas
- Lazio and Tuscany: 24 areas
- Basilicata and Apulia: 17 areas
- Sicily: 4 areas
- Sardinia: 14 areas

PUBLIC CONSULTATION: MAIN STEPS





NATIONAL SEMINAR: THE NUMBERS



SITING AND REALIZATION PROCESS



* Observations formally submitted to Sogin and MASE (former MiTE Ministry of Ecological Transition)

** CNAI will be published on the websites of Sogin, the competent Ministries and the ISIN

Security Class:

NATIONAL REPOSITORY



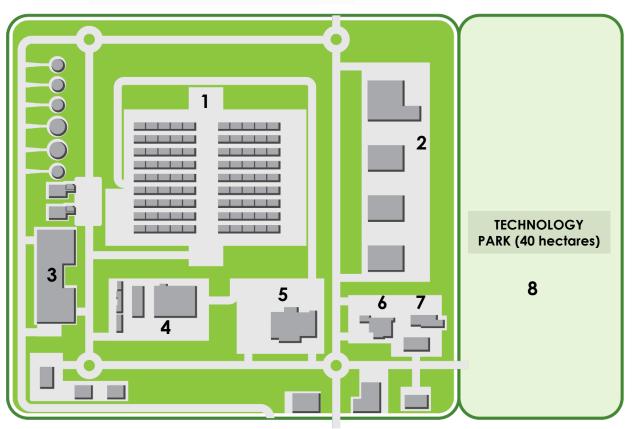


- Sogin drew up the proposal of CNAI, the National Map of Suitable Areas, based on the results of the most relevant public consultation, including a National Seminar
- On Mars 15th, 2022 Sogin has sent the proposal of the CNAI to the Ministry of Ecological Transition
- Genuary 2th, 2023 the new Ministry of Environment and Energetic Safety has sent the ISIN technical opinion to Sogin with the request of some changes and addition
- Approval of the CNAI by the Ministry of Environment and Energetic Safety in agreement with the Ministry of Infrastructures and Transports, subject to the technical opinion of the ISIN. The map shall then be published on the websites of Sogin, the two Ministries, and ISIN
- Expressions of interest of the Regions and Local Authorities having areas identified in the CNAI
- Technical investigations in the candidate sites
- Issue of the Localization Decree by the Ministry of Ecological Transition in agreement with the Ministry of Sustainable Infrastructures and Mobility

NEXT STEP FOR THE LOCALIZATION

NATIONAL REPOSITORY PRELIMINARY DESIGN

NATIONAL REPOSITORY (110 hectares)



- 1. Disposal area for VLLW and LLW
- 2. Complex for storage of ILW/HLW (Casks and High integrity containers)
- 3. Area for cell production
- 4. Module Production Plant
- 5. Module Loading Plant
- 6. Solid Waste Treatment Plant
- 7. Quality Control, Radiochemical Analyses
- 8. Technology Park with common facilities and R&D labs devoted to D&WM as well as sustainable development activities to be agreed with the local communities

Aims of Disposal



- Disposing of waste requires <u>containing and isolating</u> waste from the accessible biosphere until they represent a hazard for man and environment
- The amounts of radionuclides reaching the accessible biosphere due to any migration from the disposal facility are such that possible radiological consequences are acceptably low at all times
- During disposal the likelihood of inadvertent human intrusion into the waste is substantially reduced
- Disposal inhibits, reduces and delays the migration of radionuclides at any time from the waste to the accessible biosphere

Containment and isolation

- Containment and isolation of the waste are assured by a combination of engineered barriers in series and the natural geologic characteristics of the site
- The engineered barriers shall be designed and the host environment shall be selected so as to provide containment of the radionuclides until radioactive decay has significantly reduced the hazard posed by the waste
- Being water the main way of radionuclide transport, the barriers shall prevent water infiltration and/or leakage

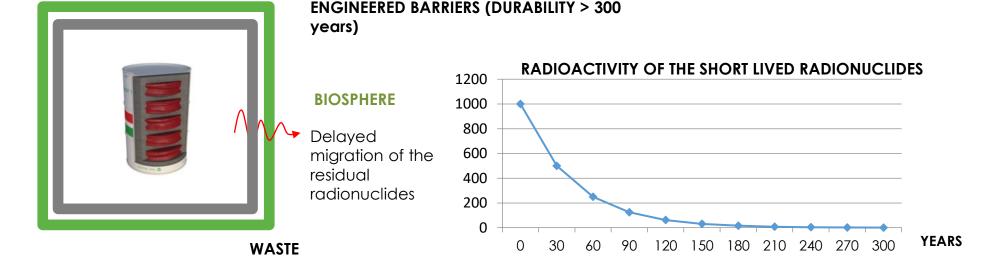


REPOSITORY

ATIONAL

Surface disposal – Basic concepts

- Limiting the inventory of the waste to be disposed of to 'short lived' waste (T/2 < 30 years; typically Cs-137 and Sr-90) and small quantities of 'long lived' RN (α emitters < 370 Bq/g)
- Engineered barriers qualified for > 300 years durability
- Institutional control period of 300 years for assuring isolation of waste
- 300 years = 10 half lives \rightarrow 1/2¹⁰ \approx 1/1000 of the initial activity of 'short lived' RN



NATURAL BARRIER (SITE GEOLOGY)

REPOSITORY

NATIONAL

DISPOSAL OF VLLW AND LLW Multiple barrier system

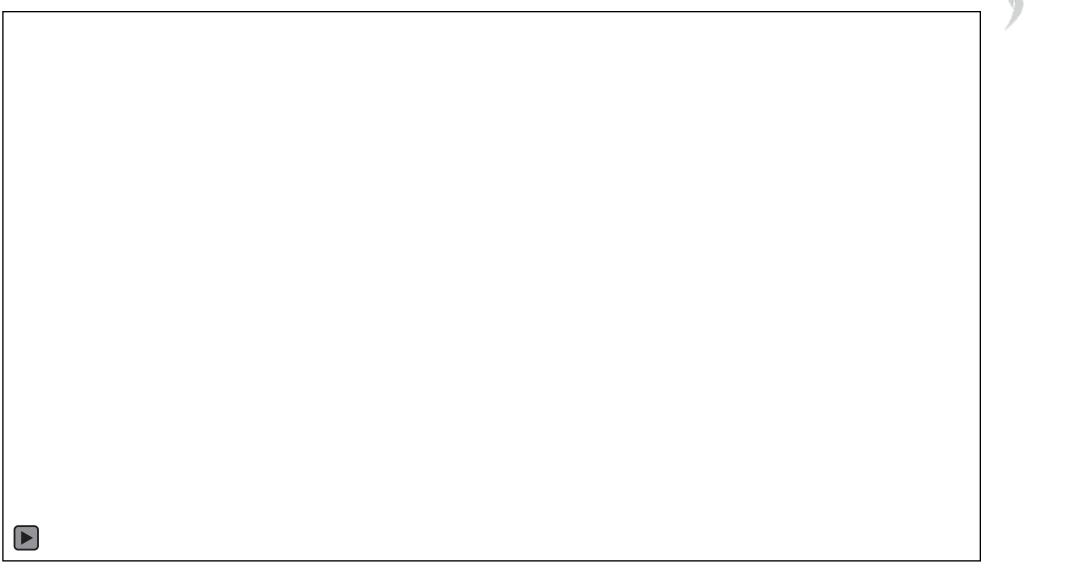




Radioactive waste conditioned with a grouting matrix inside metallic containers, transported to the National Repository

Waste packages inserted and grouted inside reinforced concrete modules (3m x 2m x 1,7m) qualified for 350 years duration 240 modules placed in a reinforced concrete vault (27m x 15,5 m x 10 m) qualified for 350 years duration Once filled with modules and sealed, the vaults are capped with a final multi-layer cover for protection against rainfalls, isolation of waste from the environment and better visual impact





EXISTING DISPOSAL SITES - FRANCE



La Manche (Normandie) Closed

NATIONAL REPOSITORY

EXISTING DISPOSAL SITES - FRANCE



L'Aube (Champagne-Ardenne) **Operating**

EXISTING DISPOSAL SITES - FRANCE

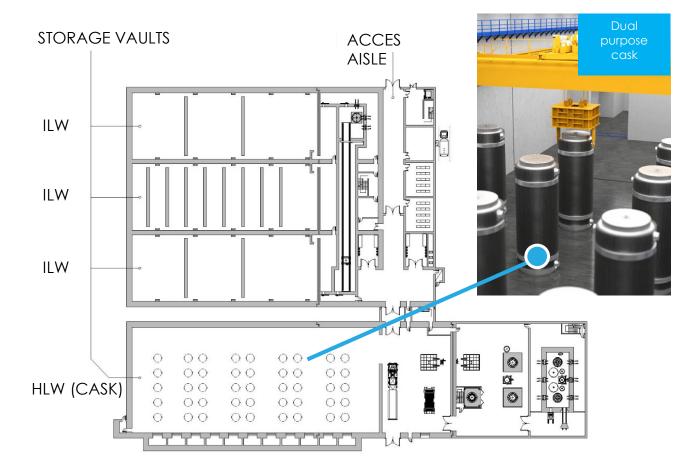




El Cabril (Provincia di Córdoba) **Operating**

STORAGE OF ILW/HLW Long-term storage Complex for ILW-HLW

- The **Complex for the long-term storage** of ILW-HLW includes **4 similar buildings**, each one with separated halls for storing different kinds of ILW. The halls are connected through an access aisle for the transfer of the ILW waste packages
- One of the buildings will host the hall dedicated to HLW (vitrified and compacted residues from fuel reprocessing) and non reprocessable irradiated fuel that will be stored in 'dual purpose' casks (high integrity and shielded containers qualified for safe transport and storage)

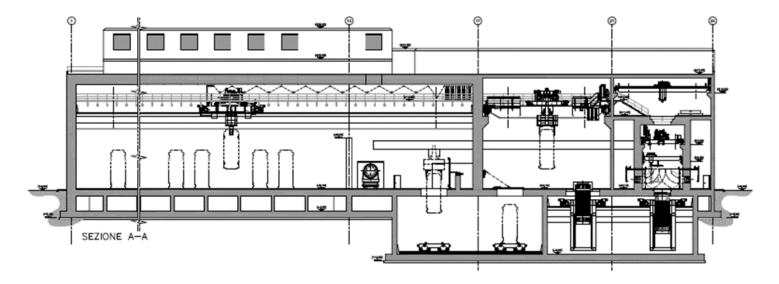


Storage building with 3 ILW halls and 1 HLW hall



HLW (CASK) STORAGE HALL

HLW (vitrified and compacted residues from fuel reprocessing) and non reprocessable irradiated fuel will be stored in a dedicated hall in 'dual purpose' casks (high integrity and shielded containers qualified for safe transport and storage). A hot cell will host the equipment for possible maintenance of the cask lids.





Dual purpose cask

REPOSITORY

NATIONAL

GORLEBEN





Zwilag



Spent fuel and vetrified residues storage building 200 cask Start operation 2000



ILW storage building 10.000 mc Start operating 2000

55/10t







Security Class: Public use

high-activity wastes.

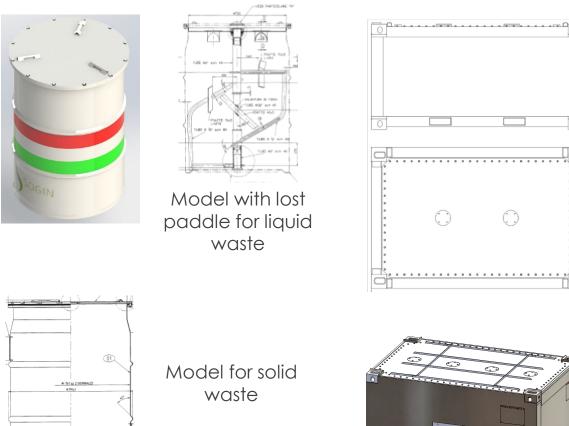
Classes: Public Use, Internal Use, Commence use, Resincted use

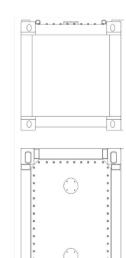
Packaging of LLW

Drum 380 I - 440 I

Box – 5,2 m3 Solid waste

Box – 2,6 m3 Solid waste









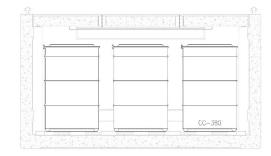




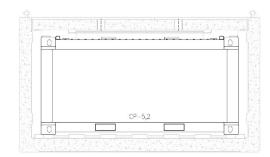
LLW packages loading into the Module

SOGIN

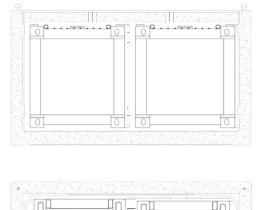
N.6 drums 380 | - 440 |

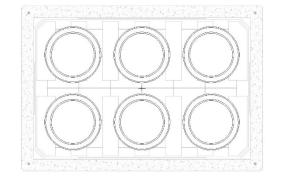


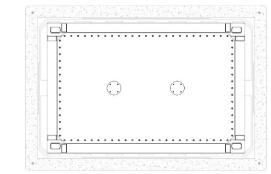
N.1 box 5,2 m3

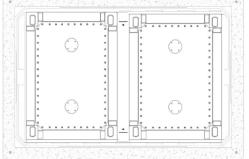


N.2 boxes 2,6 m3



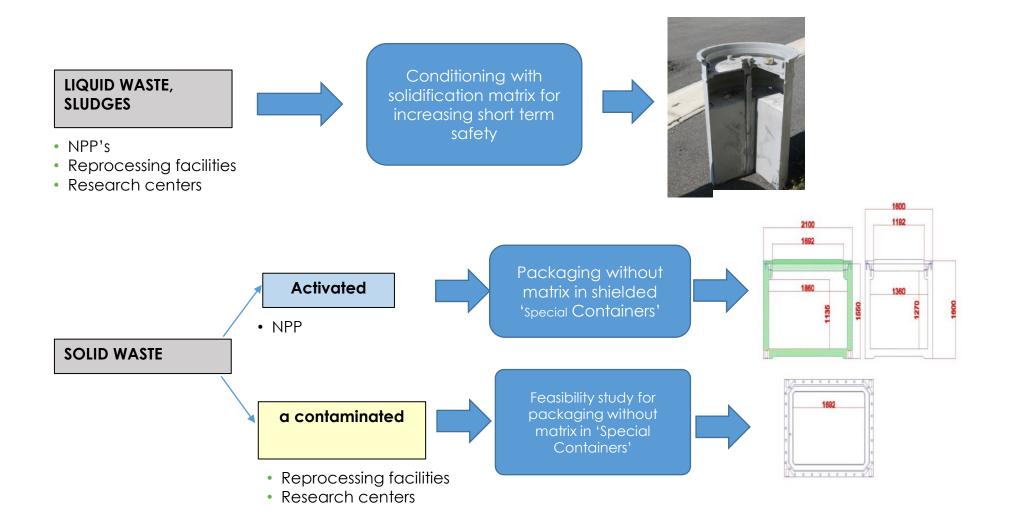






ILW packaging





NATIONAL REPOSITORY

The National Repository will be realized within a **Technology Park**, i.e. a centre of excellence for advanced R&D on nuclear matters and sustainable development with structures dedicated to information and training. It will support the local communities bringing added value to the territory

Foreseen research laboratories and infrastructures

- Laboratories on new technologies for decommissioning and waste management activities
- Laboratories for environmental analyses
- Visitors centre
- Research laboratories for sustainable development to be agreed with hosting communities during the site selection phase







The National Repository will be realized within a **Technology Park**, i.e. a centre of excellence for advanced R&D on nuclear matters and sustainable development with structures dedicated to information and training.

National and International scientific communities will be engaged in the development of the research laboratories together with the local communities

It will support the local communities bringing added value to the territory





Foreseen research laboratories and infrastructures

- Laboratories on new technologies for decommissioning and waste management activities
- Laboratories for environmental analyses
- Research laboratories for sustainable development to be agreed with hosting communities during the site selection phase
- Visitors center
- Education and training structures



Possible research fields to be explored

- New characterization systems for radioactive waste;
- New treatment and conditioning solutions for special waste (e.g. graphite);
- New monitoring systems for environment and workers
- Research activities on the Italian geological disposal





Possible research fields to be explored

- Use of nuclear science to solve problems on the conventional field
- Modification of conventional solution to be applied in nuclear field
- New solutions for renewable energy production
- New solution on the medical field
- New technologies (artificial intelligence, robotics, ...)



MAIN CHALLENGES FOR THE NATIONAL REPOSITORY REALIZATION



- The National Repository realization is a **national challenge** with a **strong local/regional dimension**. Public information, consultation and participation in environmental or technological decision-making are today's best practice
- Its implementation is an essential element to achieve the 'greenfield' status in the Italian nuclear installations, eliminating all constraints related to radiation, and to manage all radioactive waste, including that produced by industrial, research and nuclear medicine activities
- Designing and, when agreed, implementing the National Repository and Technology Park in ways that reflect the values and interests of local communities and surrounding region

National commitment and local and regional involvement are two essential dimensions of the complex task of securing continued societal agreement for the disposal of radioactive wastes

GEOLOGICAL REPOSITORY



Italian National Program

- Italy, through its research and development bodies (ENEA, CIRTEN), is participating in EU and international programs aimed at identifying an international shared geological repository (socalled "regional").
- With the start of operation of the National surface Repository, scheduled on 2029, while maintaining the primary interest in the construction, shared internationally, of a regional repository for disposal in geological formation Italy will start also studies for the identification of a national solution, to be implemented through a regulatory system aimed to provide an active involvement of the population as to that already carried out in the case of the national surface repository. The studies will be carried out also in the laboratories of the Technology Park in the National Repository
- The aim of the studies will be to define the geological repository within a further 40 years (2070), at the time of the closure of the national repository, and to allow the construction in the following 50 years (possibly also adopting an extension of the operational life of the CSA).



We protect the present We guarantee the future